

# **94-1 Core Technology Corrosion Research**

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# 94-1 Core Technology Budget

## ❖ How this work fits into 94-1 Core Technology:

- ❖ Materials ID
- ❖ Stabilization
- ➡❖ Packaging
- ➡❖ Storage
- ➡❖ Surveillance

## ❖ Customer: All sites where material will be packaged

## ❖ Current Budget \$354k

- ❖ NMT-15 (aqueous corrosion, SCC, Ga)
- ❖ MST-6 atmospheric corrosion testing
- ❖ University California Berkeley (SCC)

## ❖ Work is on schedule and on budget



# **FY 00 94-1 Core Technology Corrosion**

## **3013 Container Activities:**

- ✧ Gallium-induced failure
- ✧ Chloride-induced failure
- ✧ Shelf-life tests

## **Stress Corrosion Cracking of 3013 Containers (UCB):**

- ✧ Conclusion of work 4/30/00

## **Complex-wide Consulting :**

- ✧ MIS working group
- ✧ 3013 Standard
- ✧ Container failure at SRTC



# Corrosion Milestones / Deliverables

- ❖ Submit at least one refereed journal article (9/30/00)
- ❖ Determine atmospheric conditions within container which result in localized corrosion
  - ✧ Design, build, operate atmospheric chamber (1/31/00)
  - ✧ Perform tests using different dewpoint / temperature combinations (9/30/00)
  - ✧ Report on results (MIS meetings) and author report at finish (FY 01?)
- ❖ Establish effect of Ga on LME susceptibility of containers
  - ✧ Make servohydraulic operable (11/30/00)
  - ✧ Establish testing procedure (3/31/00)
  - ✧ Report on results (MIS meetings) and author report at finish (FY 01?)
- ❖ Shelf-life corrosion studies
  - ✧ Design, build, calibrate atmospheric corrosion rate monitors (1/31/00)
  - ✧ Perform corrosion rate studies on shelf-life packages (ongoing)
  - ✧ Report on results (MIS meetings) and author report at finish (?)



# Corrosion Milestones / Deliverables

- ❖ Assess RFETS laser welds
  - ✧ Compare corrosion performance of welded and as-received container materials (3/31/00)
  - ✧ Assess sensitization level of welds (3/31/00)
  - ✧ Compare stress corrosion cracking performance of welded and as-received container materials (9/30/00)
  - ✧ Report on results (MIS meetings) and author report (9/30/00)
- ❖ Examine the effect of stress on SCC of 316 SS
  - ✧ Produce thin film samples (4/30/00)
  - ✧ Test samples using concurrent SERS and mechanical testing (4/30/00)
  - ✧ Report on results in M.S. thesis and refereed journal articles (4/30/00)

# Background

- ❖ The problem: There is concern that storage containers will not be suitable for 50 year storage of Pu compounds.
- ❖ DOE-STD-3013-99 states that containers incorporate 30 wt% to 100 wt% Pu (< 19W). The containers may also incorporate:
  - ✧ Ga bearing compounds from weapons Pu
  - ✧ Water (up to **0.5 wt%**) / H<sub>2</sub> / O<sub>2</sub>
  - ✧ Salts / chlorinated compounds
  - ✧ Elevated temperature (Maximum excursion Temperature = **250°C**)
  - ✧ Ionizing radiation
  - ✧ Welds (inner and outer containers)
- ❖ Expected container materials:
  - ✧ Outer: 316L "Pressure Vessel". 1 or 2 welds.
  - ✧ Inner: 316 SS. Laser cut and laser welded.
  - ✧ Convenience container (for metal): 304L body, 416 lid.
  - ✧ Convenience container (for oxide): 316 body, 416 lid.



# Background

- ❖ Complex-wide experience suggests that significant problems will not occur. However, not all compounds have been stored and stored compounds have not necessarily experienced expected thermal conditions.
- ❖ Out of 113 RFETS containers, only four have shown evidence of corrosion attack.

Can No.	Packaging Date	Source of Salt	Moisture	Corrosion
D17290	4/17/80	MSE	1.5%	Rust
D77584	9/30/91	ER	1.6%	Rust
2904061	4/25/88	DOR	1.8%	Slight Rust
D37748	9/30/83	DOR	0.6%	Slight Rust

# 3013 Containers -

## Potential Failure Mechanisms

### ❖ Corrosion:

- ➡ ❖ Ga
- ➡ ❖ Cl
- ➡ ❖ H<sub>2</sub>O
  - ❖ H<sub>2</sub>O Radiolysis Products
  - ❖ Molten Salt
- ➡ ❖ Sensitization (Thermal or Radiation-Induced)
- ➡ ❖ Weld

### ❖ Embrittlement:

- ❖ Radiation
- ❖ H<sub>2</sub>
- ❖ Weld
- ❖ Transmutation

### ❖ Environmental Cracking:

- ➡ ❖ "Conventional" H<sub>2</sub>O / O<sub>2</sub> / Cl<sup>-</sup> / Cl<sub>2</sub> / HCl
  - ❖ Radiolysis
- ➡ ❖ Welds
- ➡ ❖ Ga Metal Embrittlement
  - ❖ Molten Salt
- ➡ ❖ Sensitization (Thermal or Radiation-Induced)

### ❖ Alloying:

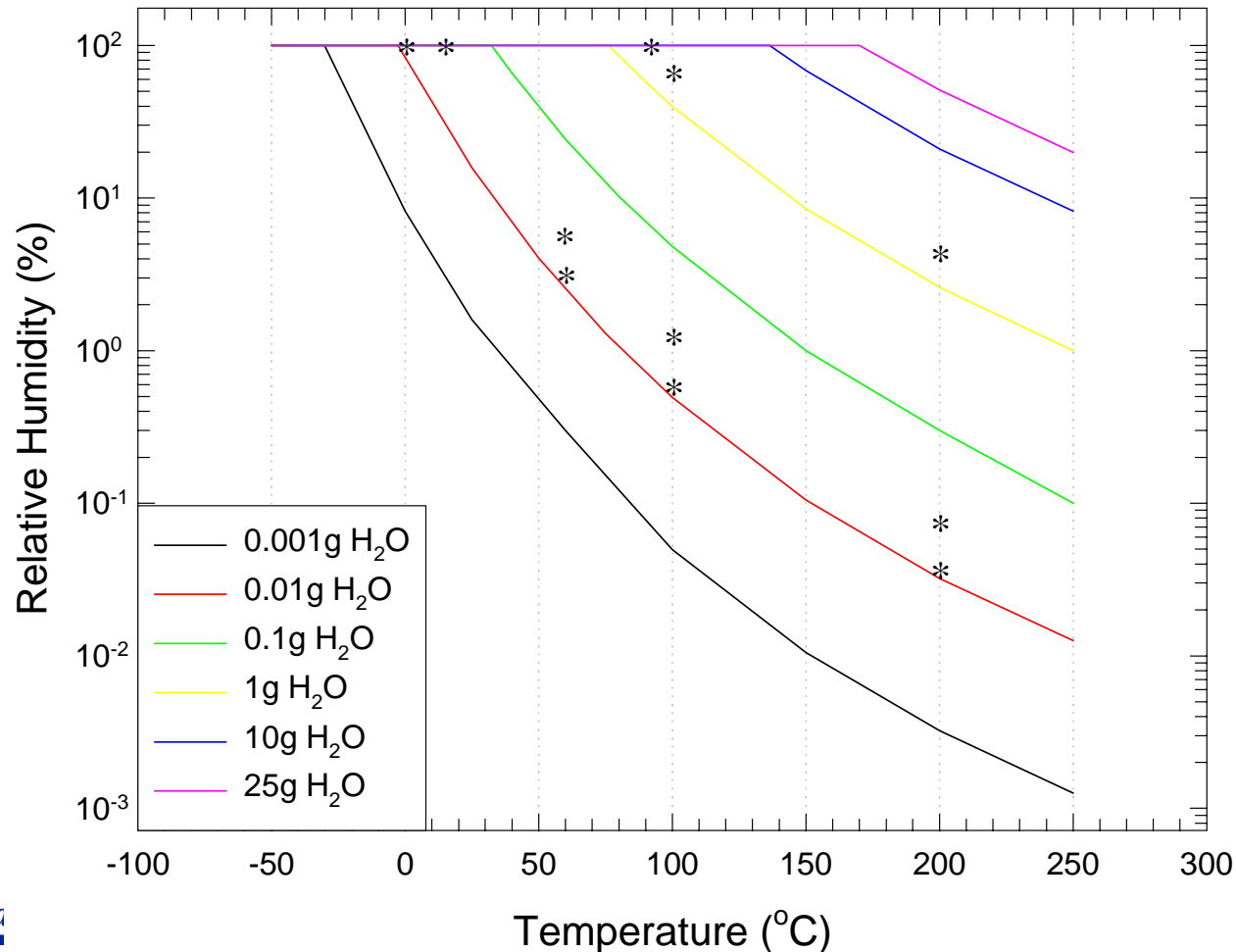
- ➡ ❖ Ga / Fe
  - ❖ Pu / Fe



# Absolute Corrosion Susceptibility

## ❖ Assuming:

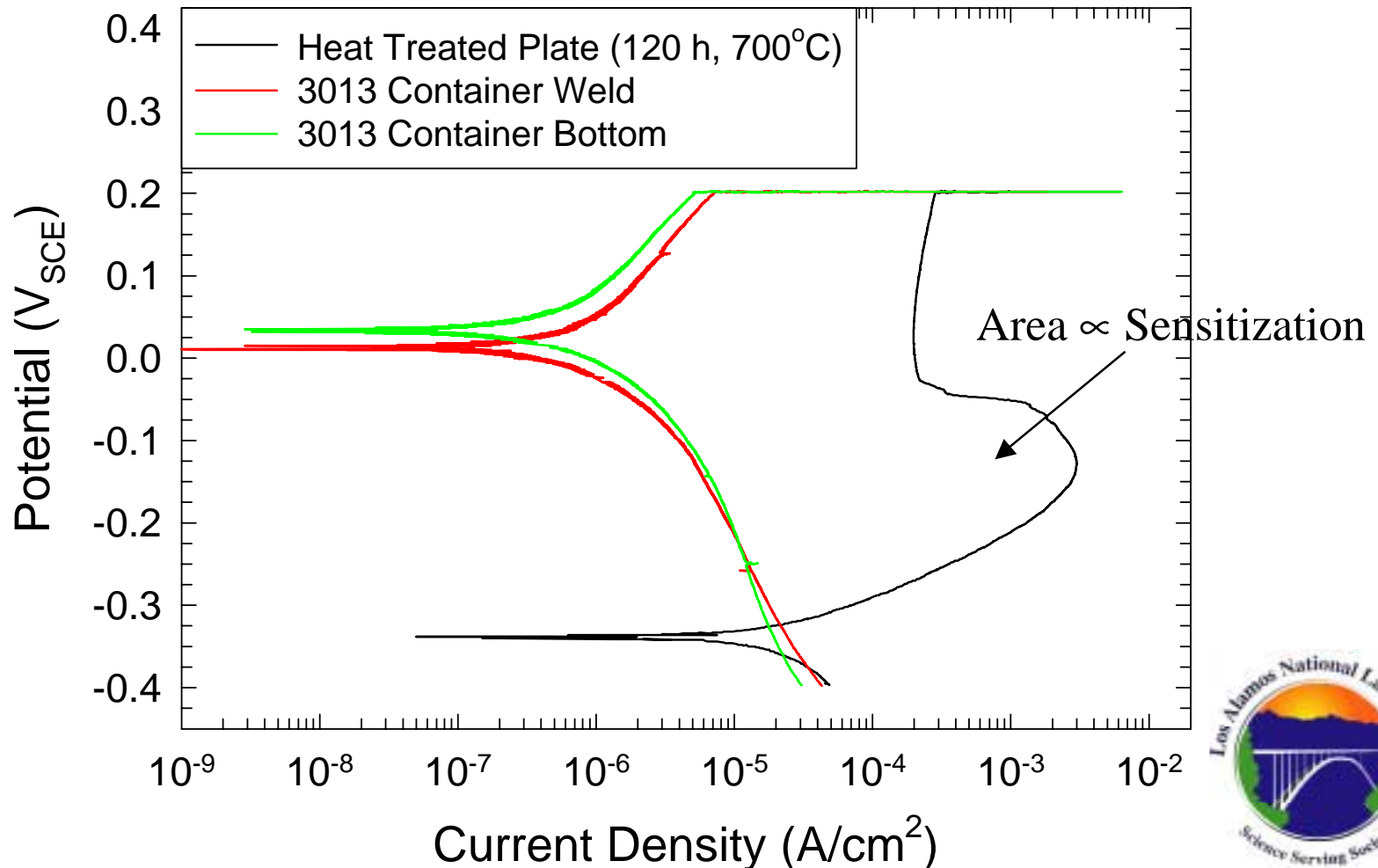
- ❖ All water is **free** to vaporize
- ❖ 2700 cm<sup>3</sup> interior volume
- ❖ LANL atmospheric pressure upon packaging



# Laser & TIG Weld Sensitization

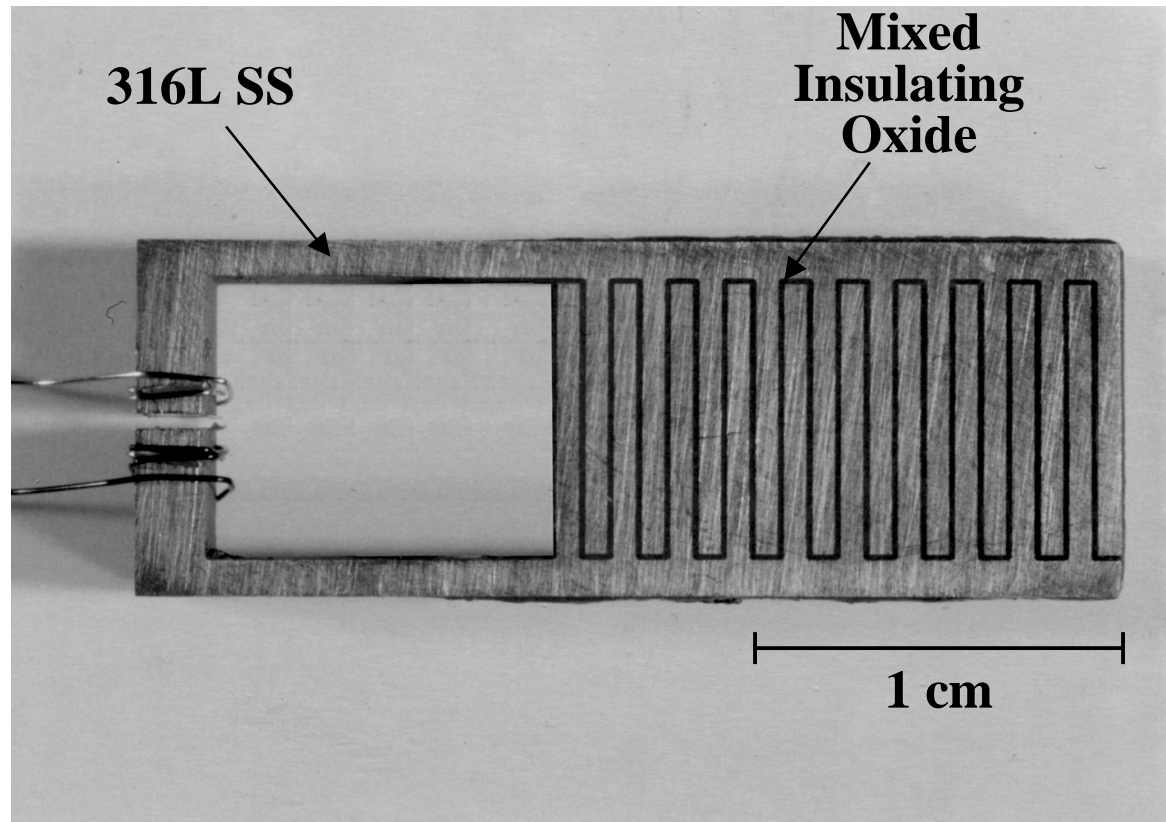
- ❖ Determine sensitization level of of RFETS laser weld using a standard test (SLEPR)

316 SS, 30°C, 0.5 M  $\text{H}_2\text{SO}_4$  + 0.01 M KSCN



# Shelf-Life: In-Situ Corrosion Monitoring

- ❖ Atmospheric corrosion rate monitor (ACRM)
  - ❖ Linear Polarization Resistance
  - ❖ Electrochemical Impedance Spectroscopy



# Stress Corrosion Cracking

❖ All of the components required for stress corrosion cracking of austenitic stainless steels are present:

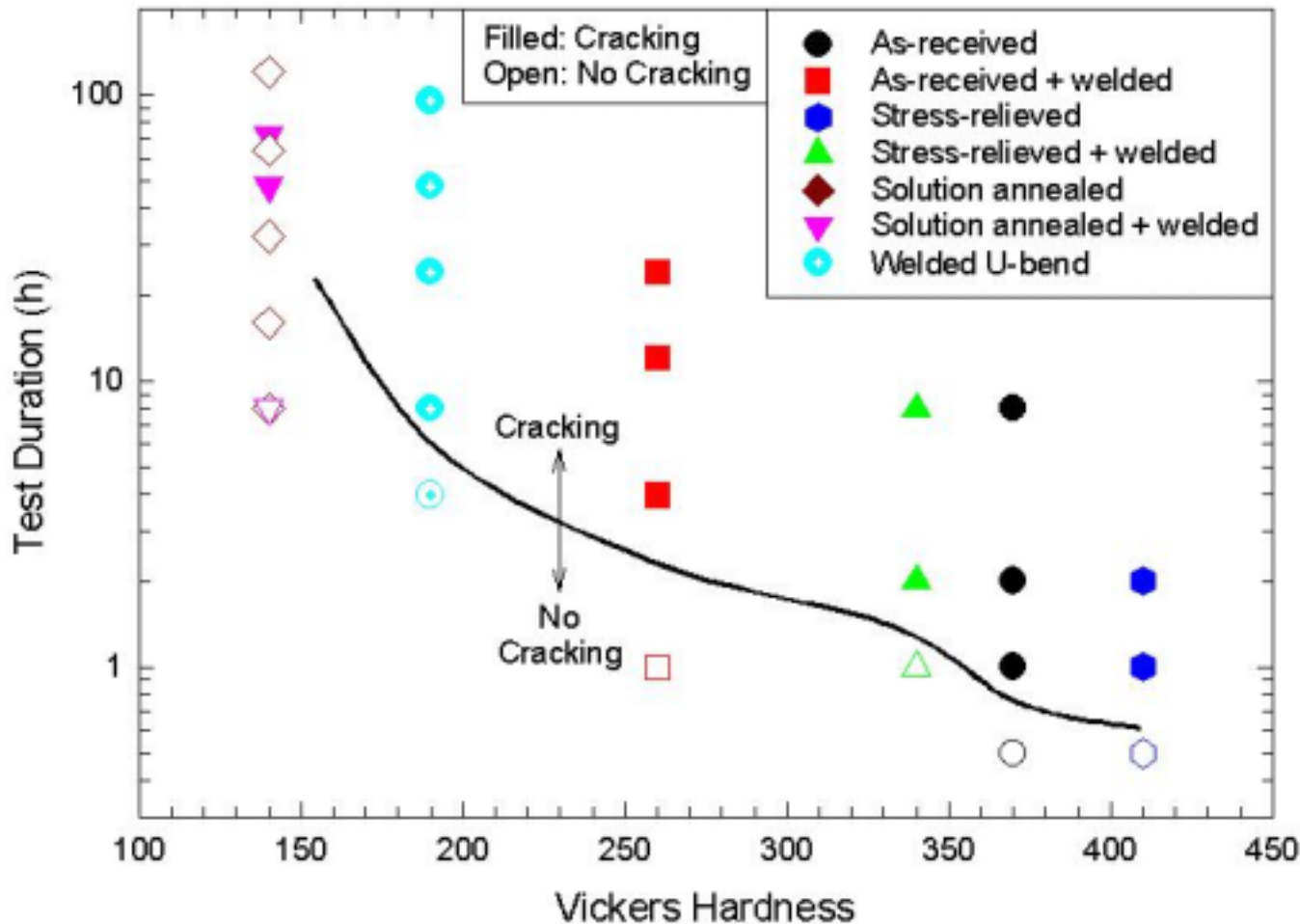
- ✧ Water (vapor and possibly liquid)
- ✧ Chloride
- ✧ Oxygen / Peroxide
- ✧ Stress

❖ Mitigated by:

- ✧ Reducing water concentration
- ✧ Reducing oxygen concentration
- ✧ Reducing stress (residual and applied)
- ✧ Avoiding dewpoint

# Relative Susceptibility to Cracking (ASTM G36)

❖ No apparent sensitization following TIG welding



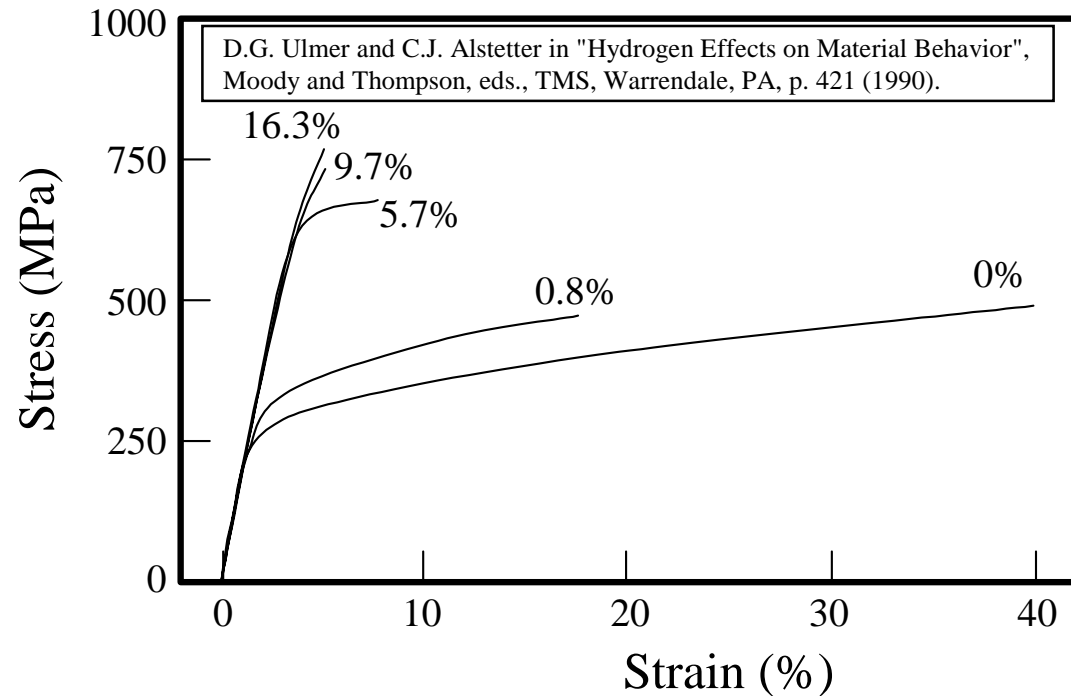
**316 SS**  
**As-received**  
**8 h**



# Embrittlement of SS

## ❖ Hydrogen Embrittlement

✧ Requires significant plastic deformation for H absorption. Therefore, H embrittlement is not a concern.



## ❖ Radiation Embrittlement

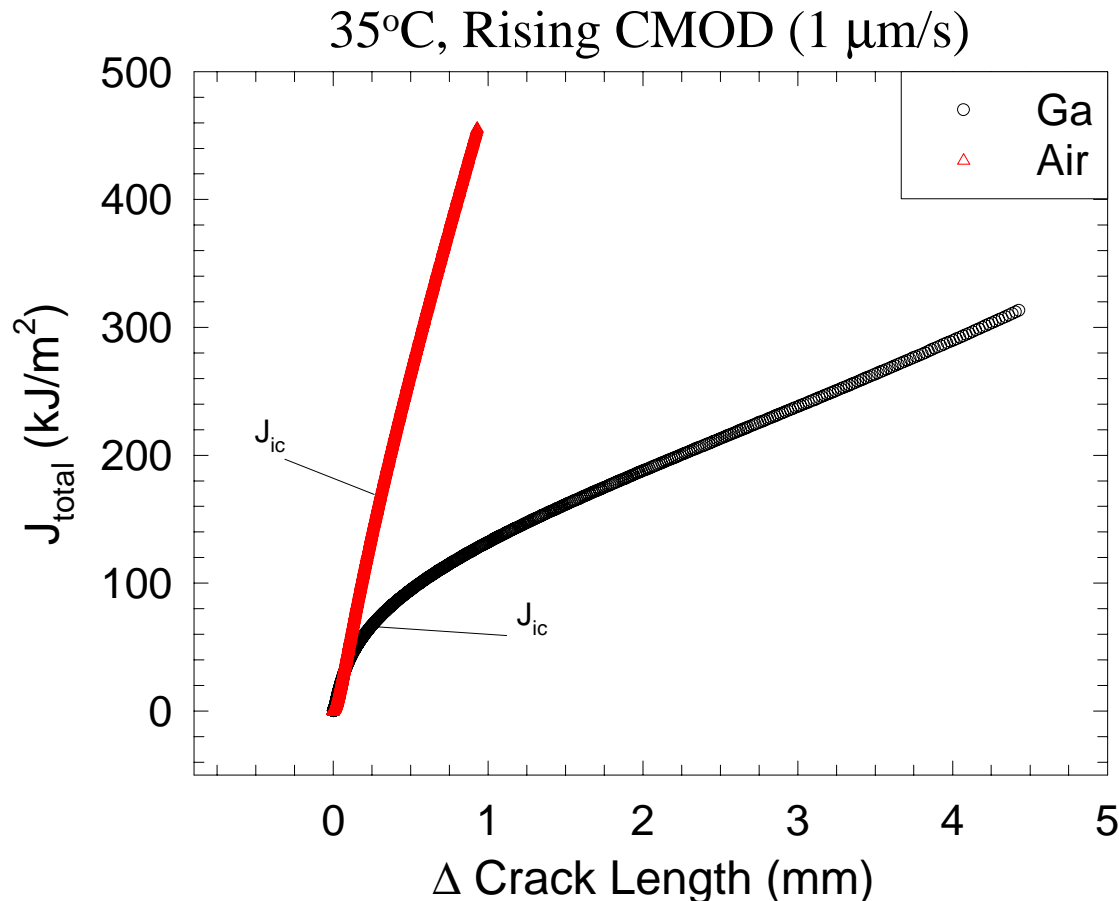
✧ Requires fluxes 10,000 times higher than that estimated over a 50 year lifetime. Therefore, radiation embrittlement is not a concern.



# Environmental Cracking Liquid Metal Ga

## ❖ Gallium-induced cracking

- ❖ Preliminary tests @ 35°C indicate that liquid metal Ga embrittles 316 L SS.
- ❖ Unstable crack growth is not observed.
- ❖ Requires Ga wetting and reduction of  $\text{Ga}_2\text{O}_3$  in the case of  $\text{PuO}_2$ .



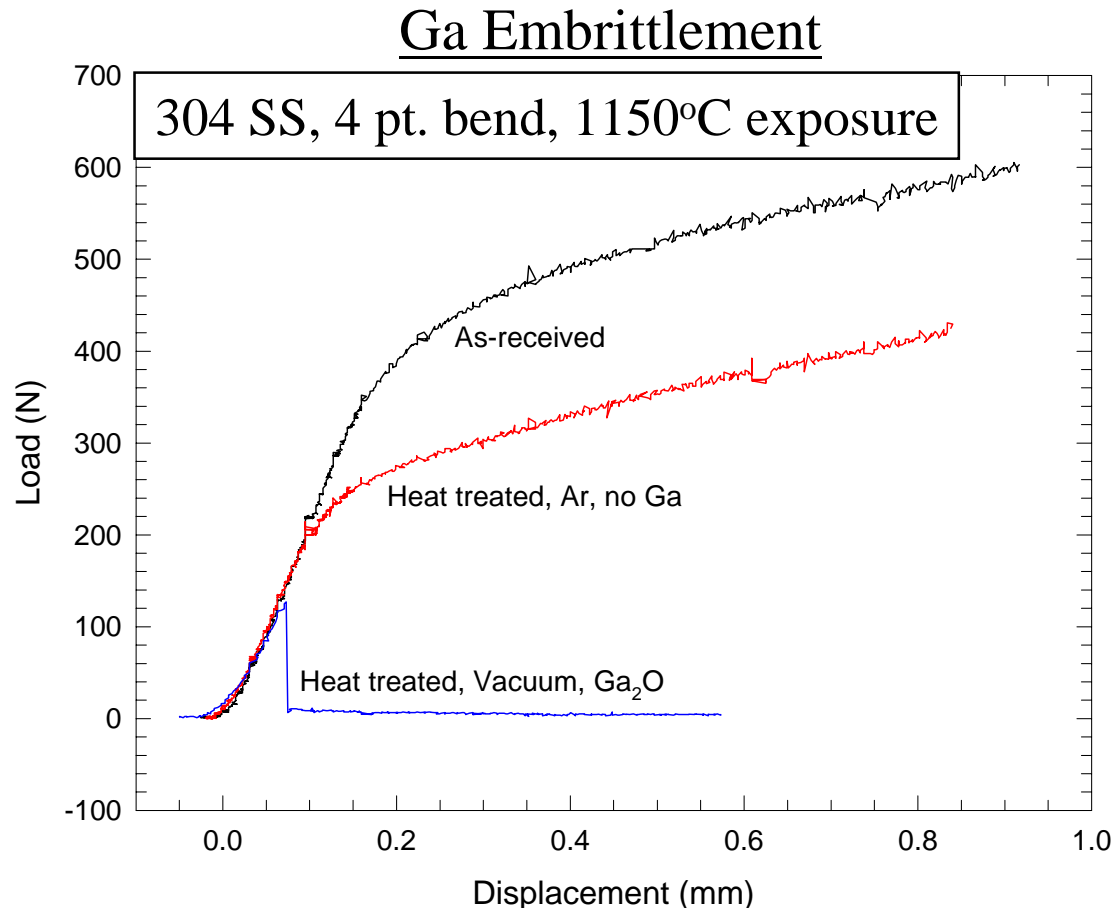
# Alloying

## ❖ Pu - Fe

✧ Based on analyses by Williamson Pu - Fe alloying is not expected to affect storage container integrity. (M.A. Williamson, "Plutonium Storage: Phase Equilibria Issues", Los Alamos National Laboratory Report # LA-UR-99-136, January 1999.)

## ❖ Fe - Ga

✧ Embrittlement of 304 SS is observed in 22°C tests following Ga uptake (30 - 50 wt%) at 1150°C (in  $\text{Ga}_2\text{O}$ ). Significant alloying of 316 SS does not appear to occur at temperatures  $\leq 200^\circ\text{C}$  (1000 h) but can occur at  $T \geq 300^\circ\text{C}$ . (L.R. Kelman et al., Argonne National Laboratory Report # ANL-4417 (1950) & P.R. Luebbbers et al., Argonne National Laboratory Report # ANL-93/31, December 1993.





# 3013 Containers -

## Potential Failure Mechanisms

### ❖ Corrosion:

- ❖ Ga
- ➡ ❖ Cl
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- ❖ Molten Salt
- ❖ Sensitization (Thermal or Radiation-Induced)
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### ❖ Embrittlement:

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### ❖ Environmental Cracking:

- ➡ ❖ "Conventional" S.C.C.:  
H<sub>2</sub>O / O<sub>2</sub> / Cl<sup>-</sup> / Cl<sub>2</sub> / HCl
- ➡ ❖ Radiolysis
- ❖ Welds
- ➡ ❖ Ga Metal Embrittlement
- ❖ Molten Salt
- ❖ Sensitization (Thermal or Radiation-Induced)

### ❖ Alloying:

- ❖ Ga / Fe
- ❖ Pu / Fe

# Conclusions-To-Date

## ❖ Corrosion:

- ❖ Experiment: Corrosion per se does not appear to be a concern. Greatest current concern is the ability of corrosion pits to nucleate stress corrosion cracks
- ❖ Experiment: Corrosion and SCC of TIG welds does not appear to be a problem.

## ❖ Embrittlement:

- ❖ Literature: Radiation flux is too low for radiation embrittlement
- ❖ Literature: Lack of significant plastic deformation obviates hydrogen embrittlement concerns
- ❖ Experiment: Ga embrittlement occurs at 35°C

# Conclusions-To-Date

## ❖ Alloying:

❖ Literature: Pu – Fe and Ga – Fe alloying does not appear to be a concern

## ❖ Stress Corrosion Cracking:

❖ Experiment: TIG welds do not have largest effect on SCC susceptibility. Largest effects are carbon content and residual stress.

❖ SCC presents the greatest threat to container integrity. All of ingredients for SCC are present and only a small amount of reactant is needed for failure. The elimination of residual stress and water is critical to avoid SCC failure.